## **APPENDIX C**

## **ABBREVIATIONS**



## LIST OF ABBREVIATIONS

ACNW	Advisory Committee on Nuclear Waste
ACR	Advanced CANDU Reactor
ACRS	Advisory Committee for Reactor Safeguards
AE	Architect/Engineer
AEC	Atomic Energy Commission
AECL	Atomic Energy of Canada, Ltd.
AGR	Advanced Gas Reactor
AHTR	Advanced High Temperature Reactor
ALARA	As Low As Reasonably Achievable
ALWR	Advanced Light Water Reactor
ANPR	Advance Notification Of Proposed Rulemaking
ANS	American Nuclear Society
AO	Abnormal Occurrence
AOO	Anticipated Operation Occurrences
AOT	Allowable/allowed outage time
ASME	American Society of Mechanical Engineers
ASP	Accident Sequence Precursor
ATHEANA	A Technique for Human Event Analysis
ATWS	Anticipated Transient Without SCRAM
AVR	Arbeitsgemeinschaft Versuchsreaktor
BDBT	Beyond Design Basis Threat
BEIR VII	Biological Effects of Ionizing Radiation
BWR	Boiling Water Reactor
BWROG	Boiling Water Reactor Owners Group
CANDU	Canadian Deuterium-Natural Uranium Reactor
C/C	Carbon/carbon-fiber composite
CCDF	Complementary Cumulative Distribution Function
CCF	Common-cause failure
CCFP	Conditional Containment Failure Probability
CCGT	Combined-Cycle Gas Turbine
CCWS	Closed Cooling Water System
	Component Cooling Water System



CD	Civil Defense
C/D	Cool-down
CDA	Containment Depressurization Actuation
	Core disruptive 'accident'
CDAs	Controls, Displays, and Alarms
CDBA	Containment Design-basis Accident
CDC	Center for Disease Control
	Computer Design Code
	Control Data Corp.
CDE	Condensate Demineralization Effluent
CDF	Core Damage Frequency
	Cumulative Damage Function
CDM	Central Data Management
	Certified Design Material
CDN	Corporate Data Network
CDP	Cask Decontamination Pit
CDPA	Civil Defense Preparedness Agency
CDP	Core damage probability
CDR	Conceptual design requirement ,
CDRG	Catastrophic Disaster Response Group
CD-ROM	Compact disk/read-only' memory
CDS	Cask Decontamination Station
	Component Disassembly Station
	Computer Data Screening
	Conceptual design study
	Condensate Demineralization Subsystem
	Current Disposal Site
CDV	Capacitance discharge vaporization
CE	Combustion Engineering Inc.
	Commonwealth Edison Co.
	Conductivity Element
	Consumer Electronics'
	Corps of Engineers, U.S. Army



C/E	Calculation/experiment
CEA	Cambridge Electron Accelerator
	Atomic Energy Commission (France)
CFC	Carbon-fiber Composite
CFR	U.S. Code of Federal Regulations
CFR 10-50	U.S. Code of Federal Regulations for Construction Permit and Operator's License
CFR 10-52	U.S. Code of Federal Regulations for Combined Construction Permit and Operator's License
СНР	Cogeneration of Heat and Power
CLB	Current Licensing Basis
CLIIP	Consolidated Line Item Improvement Process
CNS	Chem-Nuclear Systems, Inc.
СО	Carbon Monoxide
CO <sub>2</sub>	Carbon Dioxide
СР	Construction Permit
CPEF	Conditional Probability of Early Fatality
CPLF	Conditional Probability of Latent Fatality
CRCPD	Conference of Radiation Control Program Directors
CRGR	Committee to Review Generic Requirements
CRMP	configuration risk management program
CSNI	Committee on the Safety of Nuclear Installations
DBA	Design Basic Accident
DBT	Design Basic Threat
DCF	Dose Conversion Factor
DG	diesel generator
	draft guide
DOE	Department of Energy
DPO	differing professional opinion
DSI	direction-setting issue
EAB	Exclusion Area Boundary
ECCS	Emergency Core Cooling System
ECI	Emergency Cooling Injection
EDO	Executive Director of Operations



EF	Early Fatality
EIS	Environmental Impact Statement
EP	Emergency Preparedness
EPA	Environmental Protection Agency
EPIX	Equipment Performance and Information Exchange
EPRI	Electric Power Research Institute
EXF[1]	1 rem Exeedance Frequency
ESBWR	Economic and Simplified Boiling Water Reactor
ET	Executive Team
EVO	Energieversorgung Oberhausen
FAVOR	a probabilistic fracture mechanics code
FBR	Fast breeder Reactor
FC	Frequency Consequence
FCSS	Division of Fuel Cycle Safety and Safeguards (NMSS/FCSS)
FIMA	Fission per Initial Metal Ion
FSAR	Final Safety Analysis Report
FTE	Full-time Employees
F-V	Fussell-Vesely
FY	Fiscal Year
GAO	Government Accountability Office
GDC	General Design Criterion/criteria
GFR	Gas-Cooled Fast Reactor
GEM	graphical evaluation module
GIF	Generation IV International Forum
GL	generic letter
GPRA	Government Performance and Results Act
GQA	graded quality assurance
GSI	Generic Safety Issue
GTCC	Greater than Class C
GT-MHR	Gas Turbine-High Temperature Reactor
GTHTR	Gas Turbine High Temperature Reactor
HAZAP	Hazard and Operability Analysis
HC&SCC	Hydrogen Codes and Standards Coordinating Committee



HENDEL	Helium Engineering Demonstration Loop
HERA	Human Event Repository and Analysis
HEU	Highly enriched Uranium
HFIR	High Flux Isotope Reactor
HFR	High Flux Reactor
HHV	Hocktemperatur Helium Versuchanlage (High Temperature Helium Test Loop)
HIP	High Isostatic Temperature
HLR	High Level Requirement
HLW	high-level waste
HRA	human reliability analysis
HSE	Health and Safety Executive
HTE	High-temperature Electrolysis
HTGR	High Temperature Gas Reactor
HTTR	High Temperature Engineering Test Reactor
IAEA	International Atomic Energy Agency
I&C	Instrumentation and Control
ICRP	International Commission on Radiation Protection
IDCCS	Integrated Data Collection and Coding System
IEC	International Electrotechnical Commission
IEEE	Institute of Electrical and electronics Engineers
IER	Individual Early Risk
IHX	Intermediate Heat Exchanger
ILR	Individual Late Risk
IM	Importance Measure or Measures
IMC	Inspection Manual chapter
IMNS	Division of Industrial and Medical Nuclear Safety (NMSS/IMNS)
INEEL	Idaho National Engineering and Environmental laboratory
INPO	Institute of Nuclear Power Operations
INSRI	International Nuclear Energy Research Initiative
IPEEE	individual plant examination for external events
IPE	individual plant examination
ISFSI	Independent Spent Fuel Storage Installation
ISA	Integrated Safety Analysis



ISIIn-service InspectionISPRAInstitute for Environment and SustainabilityISTIn-service testingITRGIndependent Technology Review GroupJAERIJapan Atomic Energy Research InstituteJMTRJapan Materials Test ReactorKVKKomponentenversuchskreislauf (German Test Facility)LBELicensing Basis EventsLCOLimiting Conditions for OperationLERLarge Early Release FrequencyLEULow Enriched uraniumLFLatent FatalityLFRLarge Late Release FrequencyLMRLiquid metal-Cooled ReactorLOOPloss of offsite powerLPSDlow-power/shutdownLFZLow Population ZoneLRSlow-risk-significantLTCLong Term CoolingLTRLicense Termination RuleLWRLight Water ReactorMACCSMELCOR accident consequence code systemMORmonthly operating reportMOXMixed OxideMSLBMain Steam Line BreakMSPIMitigating Systems Performance IndexMW*Molten Salt Reactor	ISGTR	Induced Steam Generator Tube Rupture
ISTIn-service testingITRGIndependent Technology Review GroupJAERIJapan Atomic Energy Research InstituteJMTRJapan Materials Test ReactorKVKKomponentenversuchskreislauf (German Test Facility)LBELicensing Basis EventsLCOLimiting Conditions for OperationLERLicensee Event ReportLEVLow Enriched uraniumLFLatent FatalityLFRLaed-Cooled Fast ReactorLLRFLarge Late Release FrequencyLMRLiquid metal-Cooled ReactorLOCAloss-of-coolant accidentLOOPloss of offsite powerLP/SDlow-power/shutdownLPZLow Population ZoneLRSlow-risk-significantLTCLong Term CoolingLTRLicense Termination RuleLWRLight Water ReactorMACCSMELCOR accident consequence code systemMORmonthly operating reportMOXMixed OxideMSLBMain Steam Line BreakMSPIMitigating Systems Performance IndexMSRMolten Salt Reactor	ISI	In-service Inspection
ITRGIndependent Technology Review GroupJAERIJapan Atomic Energy Research InstituteJMTRJapan Materials Test ReactorKVKKomponentenversuchskreislauf (German Test Facility)LBELicensing Basis EventsLCOLimiting Conditions for OperationLERLicensee Event ReportLERFLarge Early Release FrequencyLEULow Enriched uraniumLFLarge Latent FatalityLFRLarge Late Release FrequencyLMRLiquid metal-Cooled ReactorLOCAloss-of-coolant accidentLOOPloss of offsite powerLP/SDlow-power/shutdownLPZLow Population ZoneLRSlow-risk-significantLTLeadership teamLTCLong Term CoolingLTRLicense Termination RuleLWRLight Water ReactorMACCSMELCOR accident consequence code systemMORmonthly operating reportMOXMixed OxideMSLBMain Steam Line BreakMSRMolten Salt Reactor	ISPRA	Institute for Environment and Sustainability
JAERIJapan Atomic Energy Research InstituteJMTRJapan Materials Test ReactorKVKKomponentenversuchskreislauf (German Test Facility)LBELicensing Basis EventsLCOLimiting Conditions for OperationLERLicensee Event ReportLERFLarge Early Release FrequencyLEULow Enriched uraniumLFLatent FatalityLFRLarge Late Release FrequencyLMRLiquid metal-Cooled ReactorLOCAloss-of-coolant accidentLOOPloss of offsite powerLPSDlow-power/shutdownLPZLow Population ZoneLRSlow-risk-significantLTLadenst Patention RuleLWRLiquid Water ReactorMACCSMELCOR accident consequence code systemMORmonthly operating reportMOXMixed OxideMSRMolten Salt Reactor	IST	In-service testing
JMTRJapan Materials Test ReactorKVKKomponentenversuchskreislauf (German Test Facility)LBELicensing Basis EventsLCOLimiting Conditions for OperationLERLicensee Event ReportLERFLarge Early Release FrequencyLEULow Enriched uraniumLFLatent FatalityLFRLaed-Cooled Fast ReactorLLRFLarge Late Release FrequencyLMRLiquid metal-Cooled ReactorLOCAloss-of-coolant accidentLOOPloss of offsite powerLP/SDlow-power/shutdownLPZLow Population ZoneLRSloss-feremination RuleLWRLicense Termination RuleLWRLight Water ReactorMACCSMELCOR accident consequence code systemMORmonthly operating reportMOXMixed OxideMSRMolten Salt Reactor	ITRG	Independent Technology Review Group
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LEULow Enriched uraniumLFLatent FatalityLFRLead-Cooled Fast ReactorLLRFLarge Late Release FrequencyLMRLiquid metal-Cooled ReactorLOCAloss-of-coolant accidentLOOPloss of offsite powerLP/SDlow-power/shutdownLPZLow Population ZoneLRSlow-risk-significantLTLeadership teamLTCLong Term CoolingLTRLicense Termination RuleLWRLight Water ReactorMACCSMELCOR accident consequence code systemMOXMixed OxideMSLBMain Steam Line BreakMSRMolten Salt Reactor	LER	Licensee Event Report
LFLatent FatalityLFRLead-Cooled Fast ReactorLLRFLarge Late Release FrequencyLMRLiquid metal-Cooled ReactorLOCAloss-of-coolant accidentLOOPloss of offsite powerLPSDlow-power/shutdownLPZLow Population ZoneLTLeadership teamLTCLong Term CoolingLTRLicense Termination RuleLWRLight Water ReactorMACCSMELCOR accident consequence code systemMOXMixed OxideMSLBMain Steam Line BreakMSRMoten Salt Reactor	LERF	Large Early Release Frequency
LFRLead-Cooled Fast ReactorLLRFLarge Late Release FrequencyLMRLiquid metal-Cooled ReactorLOCAloss-of-coolant accidentLOOPloss of offsite powerLPSDlow-power/shutdownLPZLow Population ZoneLRSlow-risk-significantLTCLong Term CoolingLTRLicense Termination RuleLWRLight Water ReactorMACCSMELCOR accident consequence code systemMORmonthly operating reportMOXMixed OxideMSLBMain Steam Line BreakMSRMolten Salt Reactor	LEU	Low Enriched uranium
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LTCLong Term CoolingLTRLicense Termination RuleLWRLight Water ReactorMACCSMELCOR accident consequence code systemMORmonthly operating reportMOXMixed OxideMSLBMain Steam Line BreakMSPIMitigating Systems Performance IndexMSRMolten Salt Reactor	LRS	low-risk-significant
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LWRLight Water ReactorMACCSMELCOR accident consequence code systemMORmonthly operating reportMOXMixed OxideMSLBMain Steam Line BreakMSPIMitigating Systems Performance IndexMSRMolten Salt Reactor	LTC	Long Term Cooling
MACCSMELCOR accident consequence code systemMORmonthly operating reportMOXMixed OxideMSLBMain Steam Line BreakMSPIMitigating Systems Performance IndexMSRMolten Salt Reactor	LTR	License Termination Rule
MORmonthly operating reportMOXMixed OxideMSLBMain Steam Line BreakMSPIMitigating Systems Performance IndexMSRMolten Salt Reactor	LWR	Light Water Reactor
MOXMixed OxideMSLBMain Steam Line BreakMSPIMitigating Systems Performance IndexMSRMolten Salt Reactor	MACCS	MELCOR accident consequence code system
MSLBMain Steam Line BreakMSPIMitigating Systems Performance IndexMSRMolten Salt Reactor	MOR	monthly operating report
MSPIMitigating Systems Performance IndexMSRMolten Salt Reactor	ΜΟΧ	Mixed Oxide
MSR Molten Salt Reactor	MSLB	Main Steam Line Break
	MSPI	Mitigating Systems Performance Index
MW <sub>e</sub> Mega-Watt Electric	MSR	Molten Salt Reactor
	MWe	Mega-Watt Electric



MW <sub>th</sub>	Mega-Watt Thermal
NDE	Non-Destructive Examination
NEI	Nuclear Energy Institute
NEPA	National Environmental Policy Act
NERI	Nuclear Energy Research Institute
NFI	Nuclear Fuel Industries
NFPA	National Fire Protection Association
NGNP	Next Generation Nuclear Plant
NMSS	NRC Office of Nuclear Material Safety and Safeguards
NOAK	n <sup>th</sup> -of-a-kind
NOED	Notice of Enforcement Discretion
NPP	Nuclear Power Plant
NRC	Nuclear Regulatory Commission
NRS	non-risk-significant
NRR	NRC Office of Nuclear Reactor Regulation
NTPBMR	Nuclear Technology Pebble Bed Modular Reactor
NSSS	Nuclear Steam Supply System
OAS	Organization of Agreement States
OCFO	NRC Office of the Chief Financial Office
ODS	Oxide Dispersion Strengthened
OEDO	NRC Office of the Executive Director for Operations
OKBM	Experimental Designing Bureau of Machine Building (Russia)
O&M	Operation and Maintenance
OL	Operating License
OGC	Office of General Council
OSTP	NRC Office of State and Tribal Programs
PA	Performance Assessment
PAG	Protective Action Guidelines
PBMR	Pebble Bed Modular Reactor
PBPM	Planning, Budgeting, and Performance Management
PCHEs	Printed Circuit Heat Exchangers
РСТ	Peak Cladding Temperature
PCU	Power Conversion Unit



PIE	Postulated Initiating Event
PNP	Prototypanlage Nukleare Prozesswaerme (Prototype plant Nuclear Process Heat)
PRA	Probabilistic Risk Assessment
PRASC	PRA steering committee
PRM	Petition for Rulemaking
PSAR	Preliminary Safety Analysis Report
PTS	Pressurized Thermal Shock
Pu	Plutonium
PV	Pressure Vessel
PWR	Pressurized-Water Reactor
QA	Quality Assurance
QC	Quality Control
QHO	Quantitative Health Objectives
RAD	Radiation Absorption Dose
RADS	Reliability and Availability Data System
RASP	Risk Assessment Standardization Project
RAW	Risk Achievement Worth
RBI	risk-based performance indicator
RCCS	Reactor Cavity Cooling System
RCS	Reactor Coolant System
R&D	Research and Development
REM	Roentgen Equivalent Man
RES	NRC Office of Nuclear Regulatory Research
RG	Regulatory Guide
RI	Risk Informed
RIE	Risk Informed Environment
RILP	Risk Informed Licensing Panel
RIPB	Risk Informed, Performance Based
RIRIP	Risk Informed Regulation Implementation Plan
RIS	Regulatory Issue Summary
ROP	Reactor Oversight Process
RPM	Revolutions per Minute
RPV	Reactor Pressure Vessel



RTG	Risk Task Group (NMSS)
SAMDA	Severe Accident Mitigation Design Alternative
SAR	Safety Analysis Report
SAPHIRE	Systems Analysis Program for Hands-on Integrated Reliability Evaluation
SBO	station blackout
SCRAM	super critical reactor axe man
SCSS	sequence coding and search system
SCWR	Super Critical Water Reactor
SDP	significance determination process
SFPO	Spent Fuel Project Office (NMSS)
SFR	Sodium-Cooled Fast Reactor
SI	Sulfur-iodine
SG	Steam Generator
SGTAP	Steam Generator Task Action Plan
SGTR	Steam Generator Tube Rupture
SMR	Steam Methane Reforming
SNM	Special Nuclear Material
SPAR	Standardized Plant Analysis Risk
SR	Stress Relief
SRM	Staff Requirements Memorandum
SRP	Standard Review Plan
SSC	Systems, Structures and Components
STP	South Texas Project
STS	Standard Technical Specifications
TBD	to be determined
ТІ	temporary instruction
TID	Technical Information Document
ТМІ	Three Mile Island
ТР	Total Population
TRISO	Ceramic-coated-particle fuel
TS	Technical Specification
TSTF	Technical Specification Task Force
ттс	NRC Technical Training Center



UAI(system) unavailability indexUCOUranium Oxycarbide
·
UL Underwriters Laboratories
URI (system) unreliability index
USI Unresolved Safety Issue
VHTR Very High Temperature Reactor
V&V Verification and Validation
WOG Westinghouse Owners Group